



EMC3-EIRENEによる 周辺プラズマ・不純物輸送モデリング

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- ◆ EMC3-EIRENEによる周辺プラズマモデリング
- ◆ LHDにおける周辺プラズマ解析
- ◆ タングステンダイバータ損耗再堆積解析
- ◆ 直線装置プラズマモデリング



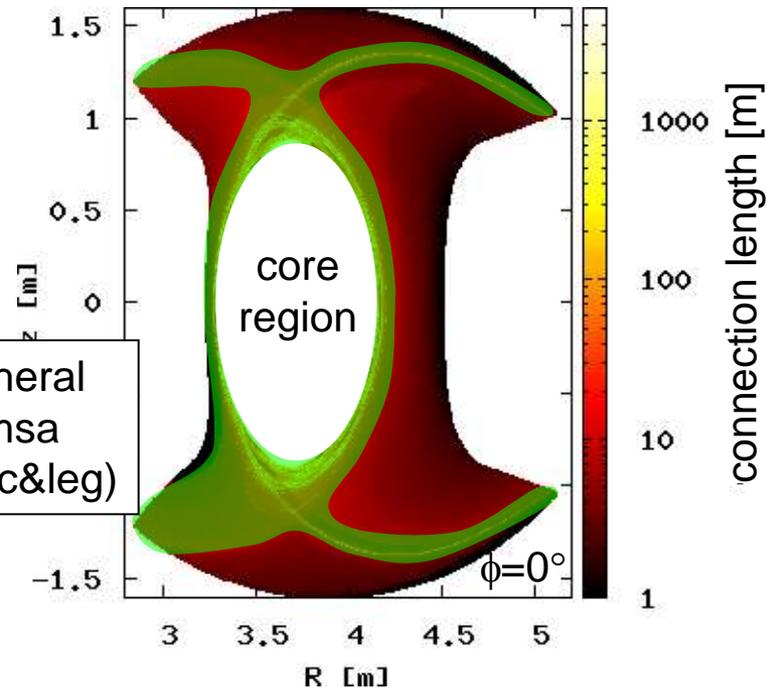
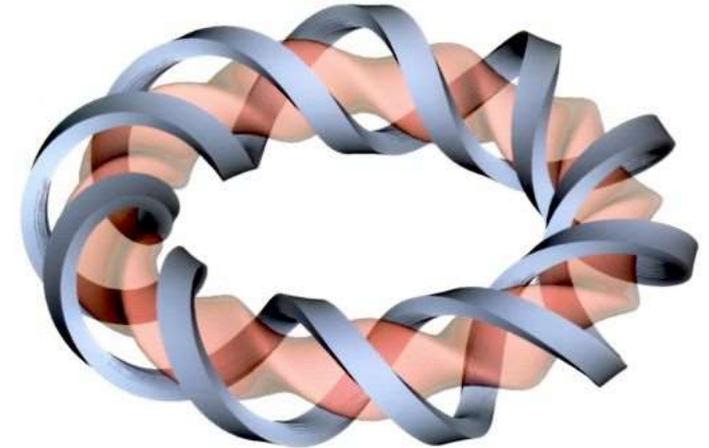
Transport simulation of LHD peripheral plasmas

◆ Background and Motivations

- Understandings of **plasma-neutral-impurity interactions** and realization of predictive simulation.
- Evaluation of **different divertor configuration**; open and closed.

◆ Approach

- Realization of **three-dimensional** transport simulation in **ergodic and leg** regions in the realistic geometry.
- Development of a three-dimensional transport code for LHD; **EMC3-EIRENE**
- Validation by measurements.
- Comparisons of results for open/closed divertor configurations.





EMC3-EIRENE code

- ◆ EMC3 solves fluid equations of flux tube plasma in SOL:

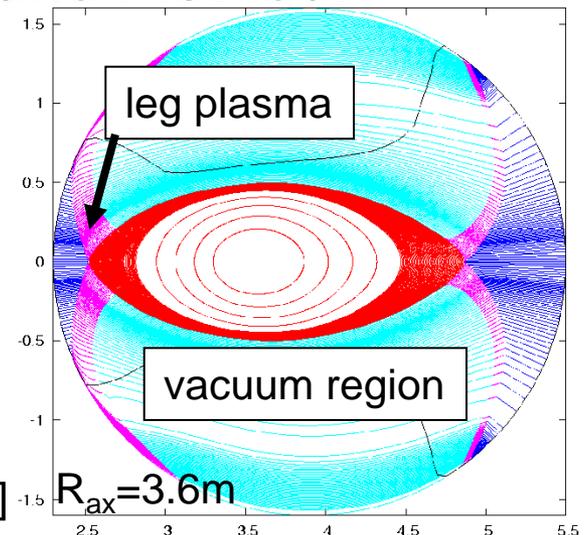
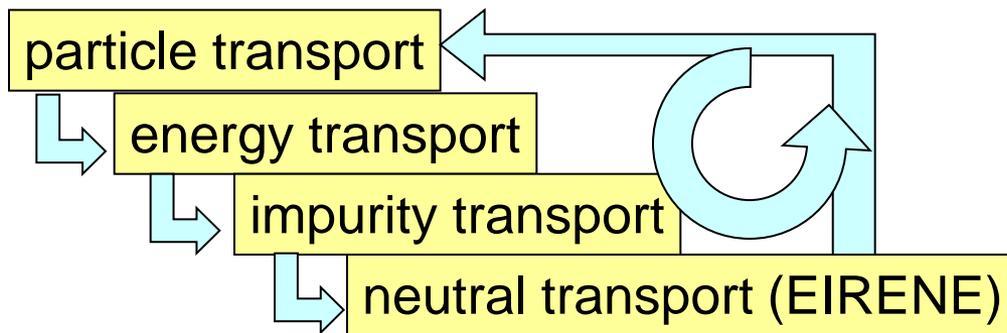
$$\nabla_{\parallel} \cdot (n\mathbf{v}_{\parallel}) + \nabla_{\perp} \cdot (-D\nabla_{\perp} n) = S_p$$

$$\nabla_{\parallel} \cdot (m_i n \mathbf{v}_{\parallel} \mathbf{v}_{\parallel} - \eta_{\parallel} \nabla_{\parallel} \mathbf{v}_{\parallel}) + \nabla_{\perp} \cdot (-m_i \mathbf{v}_{\parallel} D\nabla_{\perp} n - \eta_{\perp} \nabla_{\perp} \mathbf{v}_{\parallel}) = \nabla_{\parallel} p + \mathbf{S}_m$$

$$\nabla_{\parallel} \cdot \left(-\kappa_i \nabla_{\parallel} T_i + \frac{5}{2} n T_i \mathbf{v}_{\parallel} \right) + \nabla_{\perp} \cdot \left(-\chi_i n \nabla_{\perp} T_i - \frac{5}{2} T_i D\nabla_{\perp} n \right) = k(T_e - T_i) + S_{ei}$$

$$\nabla_{\parallel} \cdot \left(-\kappa_e \nabla_{\parallel} T_e + \frac{5}{2} n T_e \mathbf{v}_{\parallel} \right) + \nabla_{\perp} \cdot \left(-\chi_e n \nabla_{\perp} T_e - \frac{5}{2} T_e D\nabla_{\perp} n \right) = -k(T_e - T_i) + S_{ee}$$

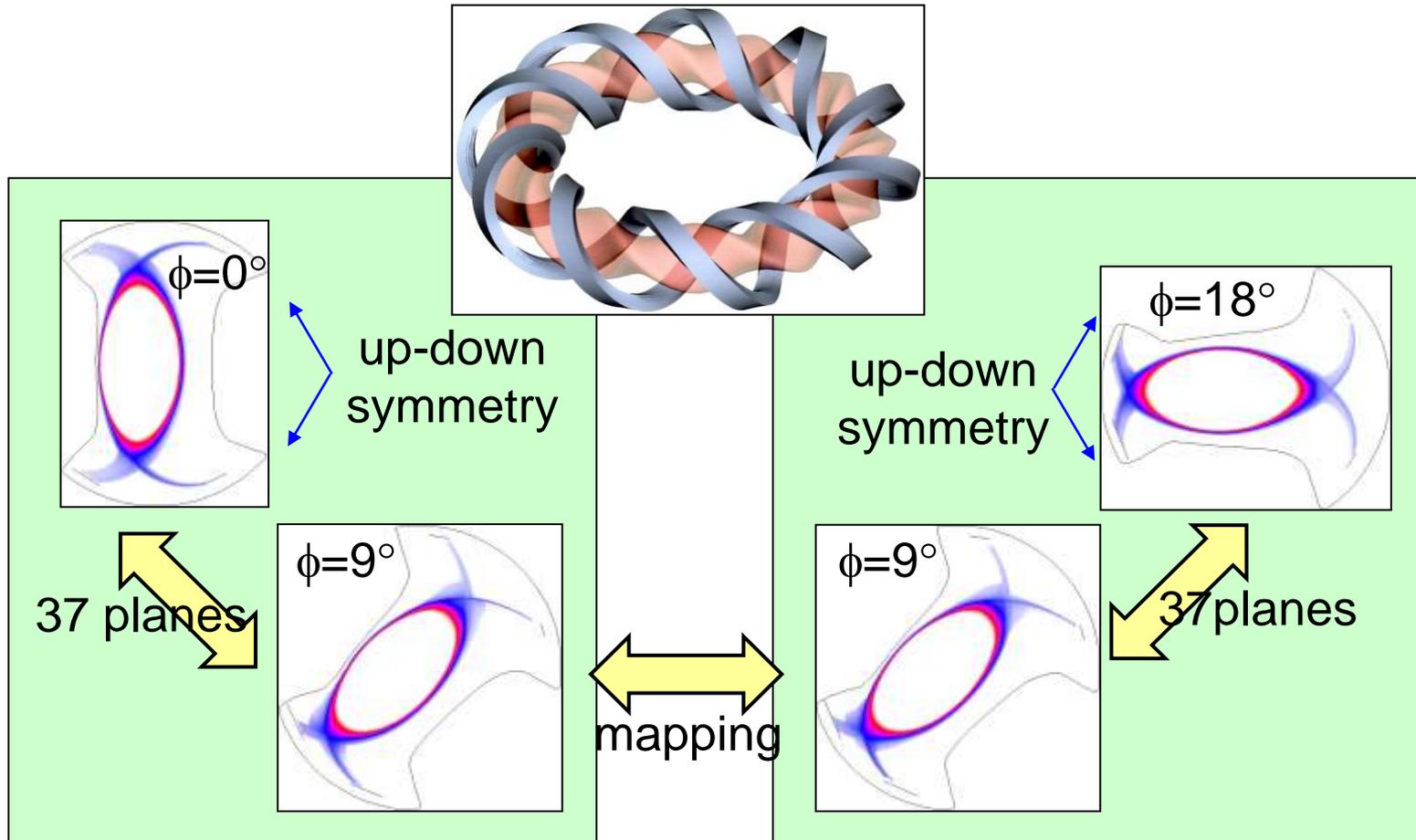
- ◆ EMC3 can simulate plasma with ergodic magnetic field in LHD by solving them with the aid of Monte Carlo method.





Simulation box

- ◆ 18° -section (1/20 of the torus) is realized in simulation by using the periodicity and assumption of up-down symmetry.
- ◆ Meshes are generated on poloidal planes every 0.25° .





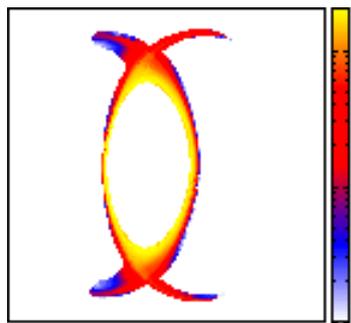
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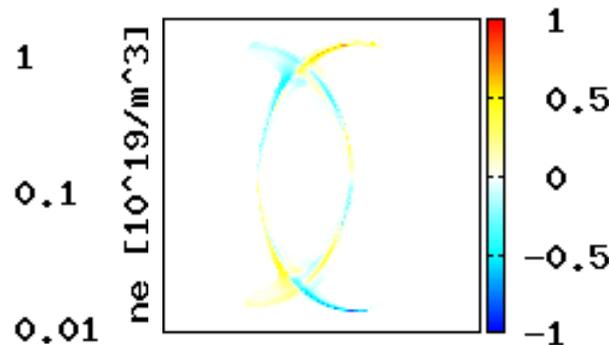
Simulation conditions and examples

◆ Conditions and Assumptions

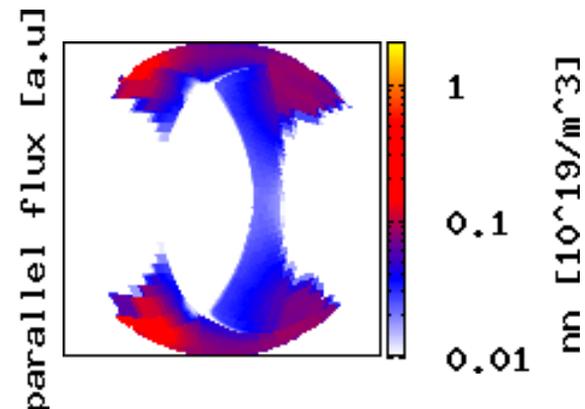
- Input power: 8MW
- Electron density: $2 \times 10^{13}/\text{cm}^3$ at LCFS
- Bohm condition is assumed at divertor plates
- Magnetic configuration: $R_{ax}=3.6\text{m}$
- Perpendicular transport coefficient: $D_{\text{perp}}=1 \text{ m}^2/\text{s}$, $\chi_{\text{perp}}=3 \text{ m}^2/\text{s}$
- No neutral-pumping (100% recycled)



electron density
[$10^{17} - 2 \times 10^{19} / \text{m}^3$]

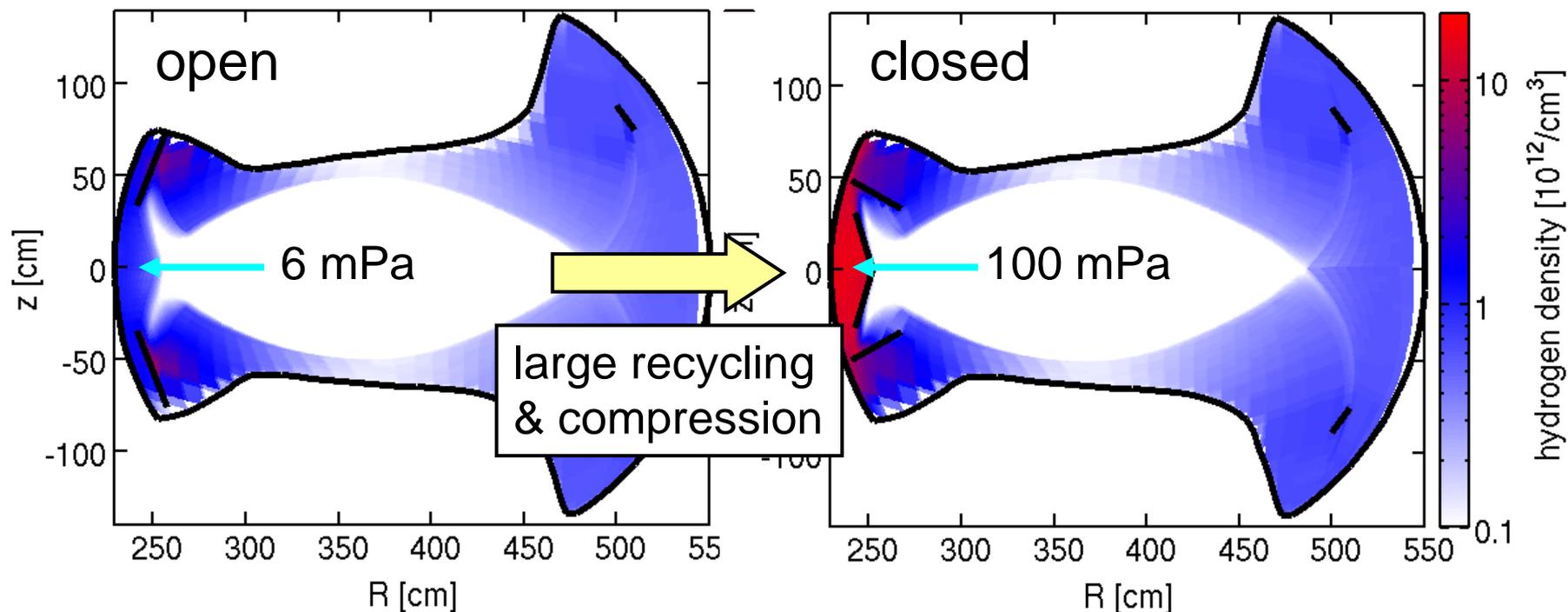


parallel particle flux
[a.u.]



H₂ molecule density
[$10^{17} - 10^{19} / \text{m}^3$]

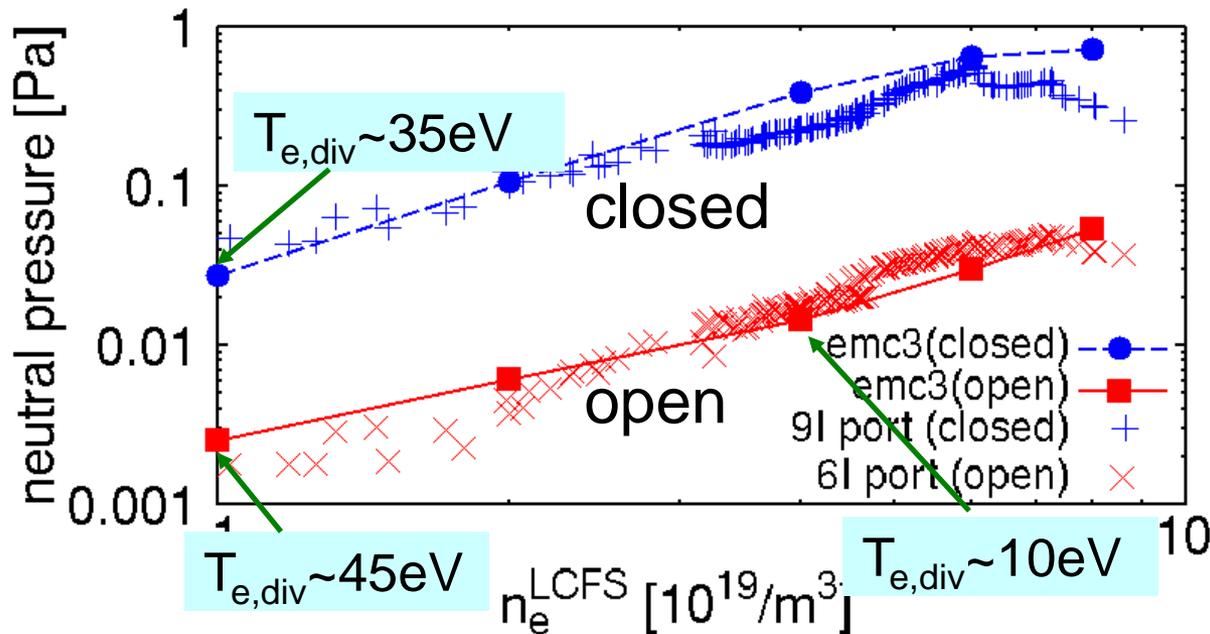
- ◆ Influence of the closed configuration on neutral-gas pressure
 - Compression of hydrogen gas
 - Simulation gives 17 times higher density under the dome structure.
 - It agrees with experimental measurement (10~20 times)





Scaling of neutral pressure with electron density

- ◆ A series of simulation were carried out.
 - Constant input power: 8MW, n_e at LCFS: 1, 2, 4, 6, $8 \times 10^{13}/\text{cm}^3$.
- ◆ Dependence of neutral pressure on electron density.
 - Simulation results has the same scaling law as the experimental measurements independently of configurations.

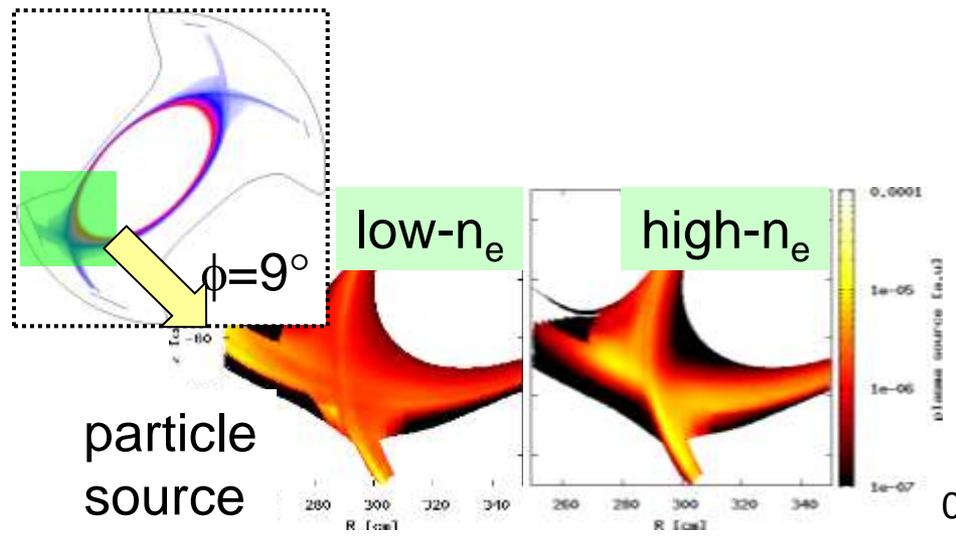
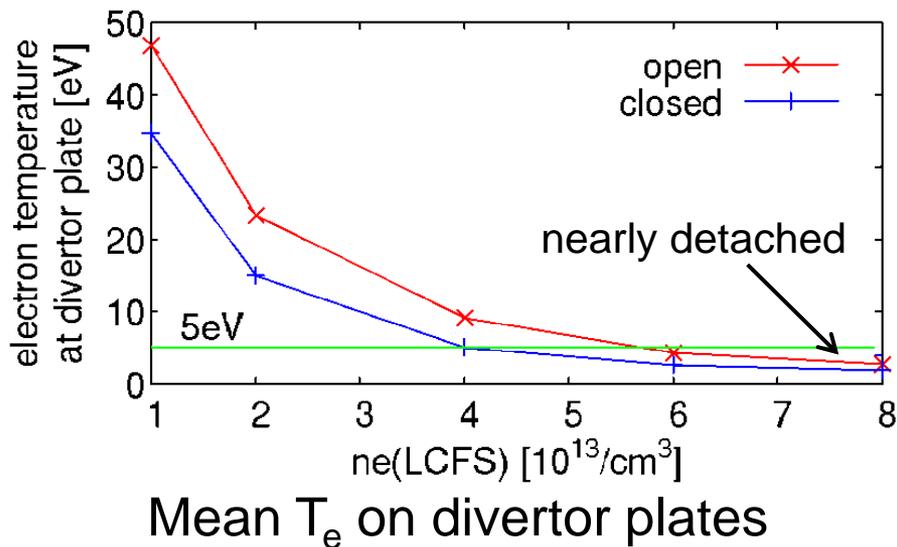


Comparison of neutral pressure; measurements of two discharges and simulation results (solid lines: 500K H₂ gas)



Hydrogen recycling

- ◆ Recycling in open/closed configuration
 - Closed configuration causes large particle source, i.e. large recycling.
 - Large ionization leads to large energy loss.
- ◆ Contribution of divertor legs on recycling
 - Major contribution in low-density discharges, $1\sim 4\times 10^{13}/\text{cm}^3$
 - Minor contribution in high-density discharges, $6\sim 8\times 10^{13}/\text{cm}^3$





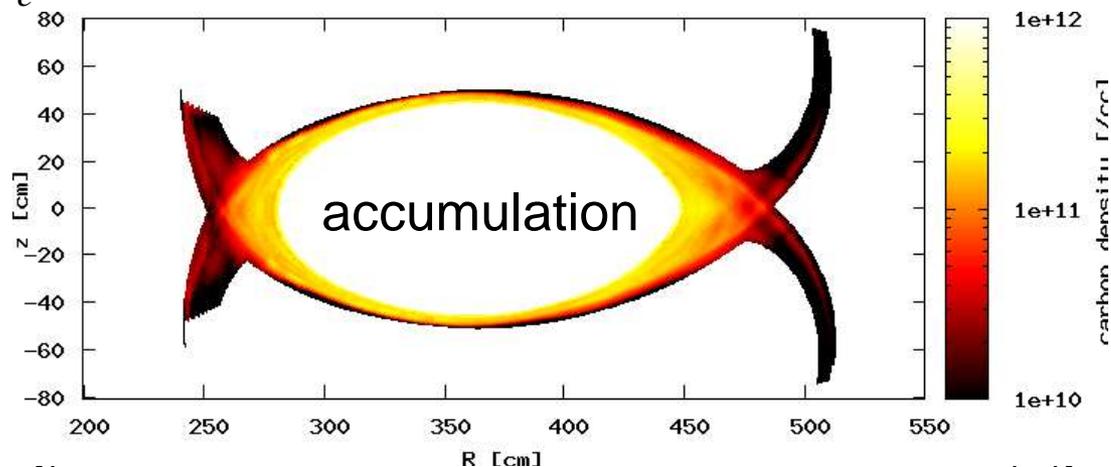
Impurity transport

- ◆ Impurity screening [M. Kobayashi et al., JNM 390 (2009) 325]

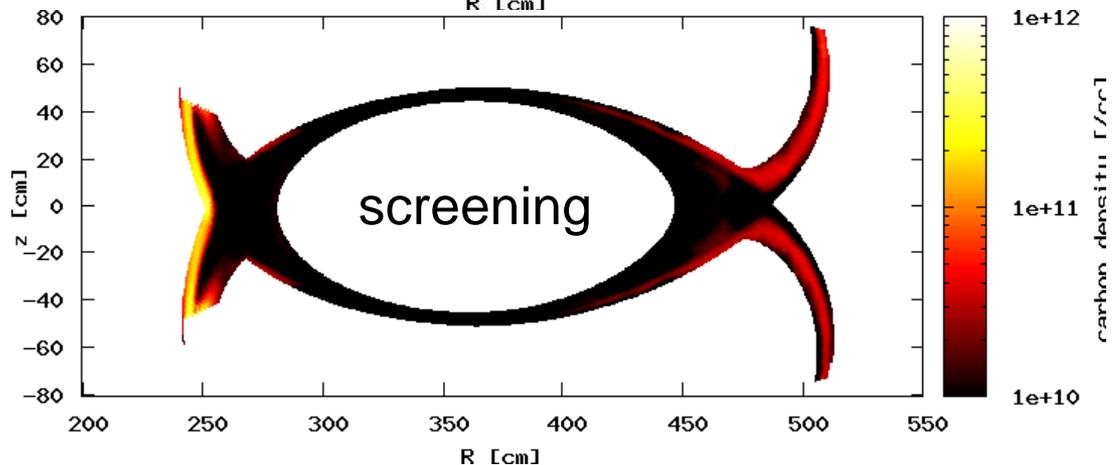
Friction force $\propto v_{\parallel}$ \rightarrow divertor plates

Thermal force $\propto \nabla_{\parallel} T_e$ \rightarrow core

Low-density discharge:
 $n_e(\text{LCFS}) = 1 \times 10^{13} / \text{cm}^3$



Hight-density discharge:
 $n_e(\text{LCFS}) = 6 \times 10^{13} / \text{cm}^3$

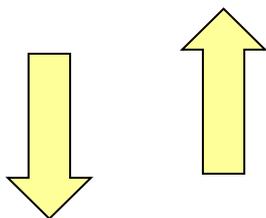




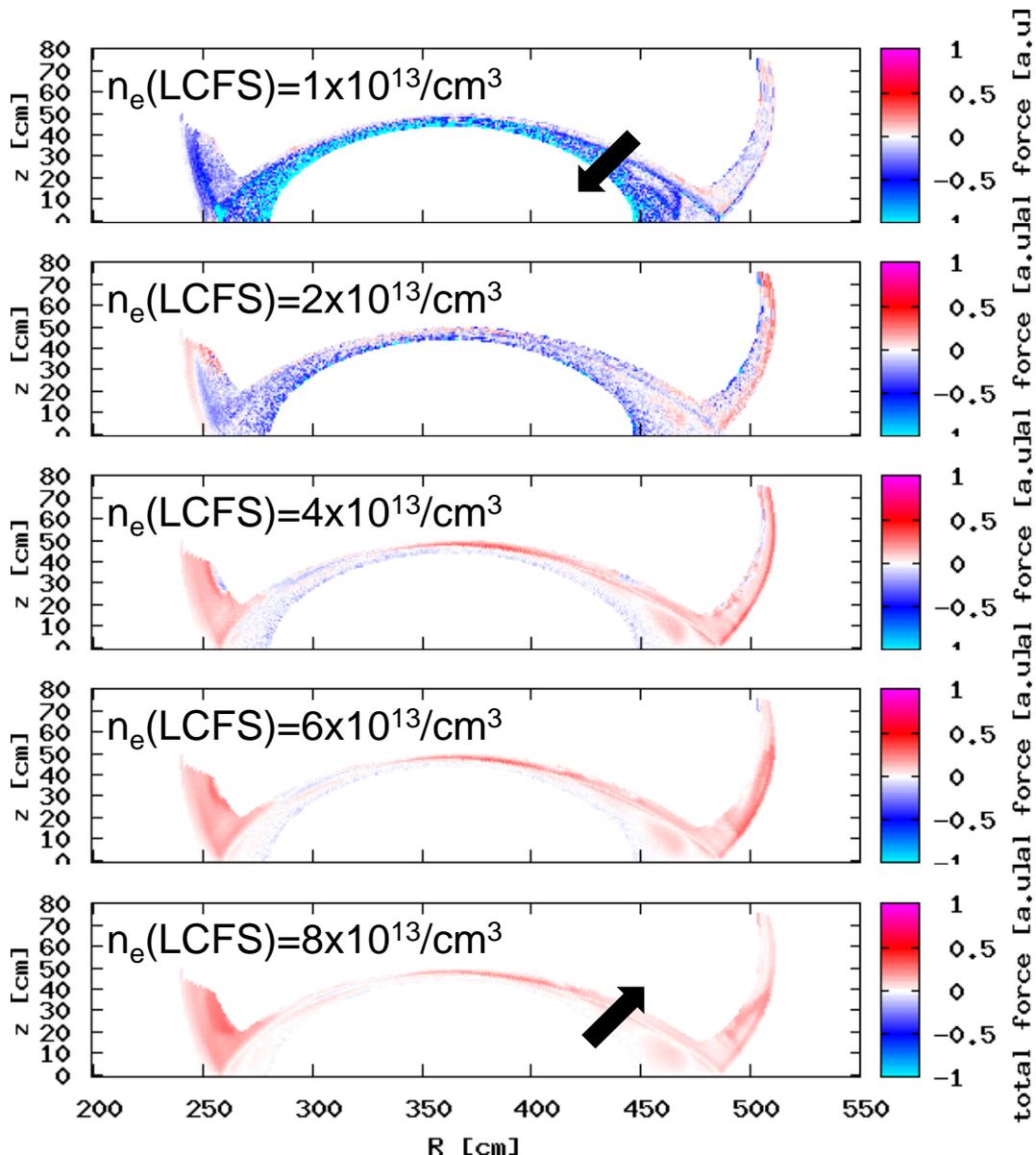
Force balance on impurity ions

◆ Friction force vs. thermal force

thermal force dominant
toward the upstream



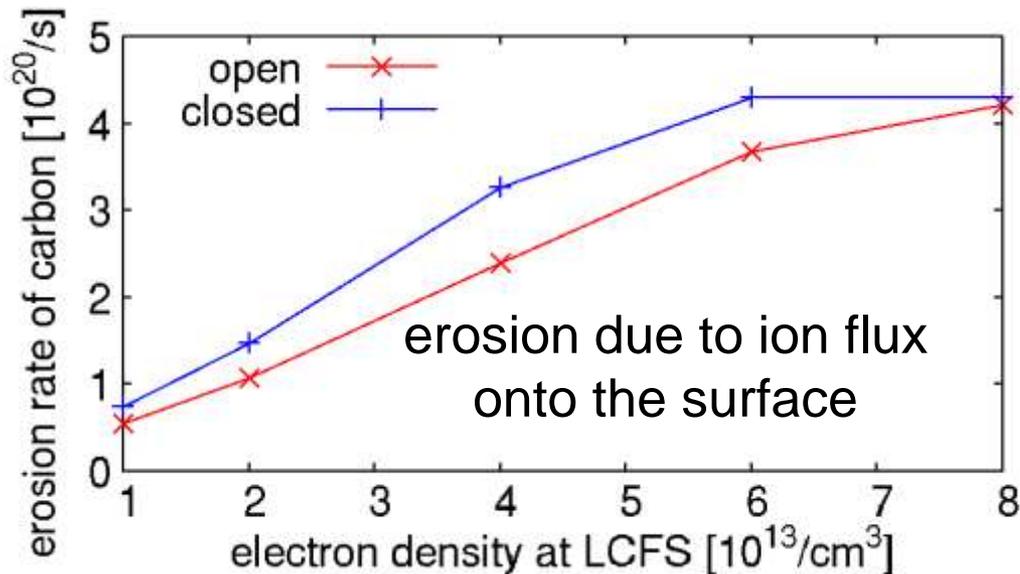
friction force dominant
toward the downstream





Erosion rate of the divertor plates

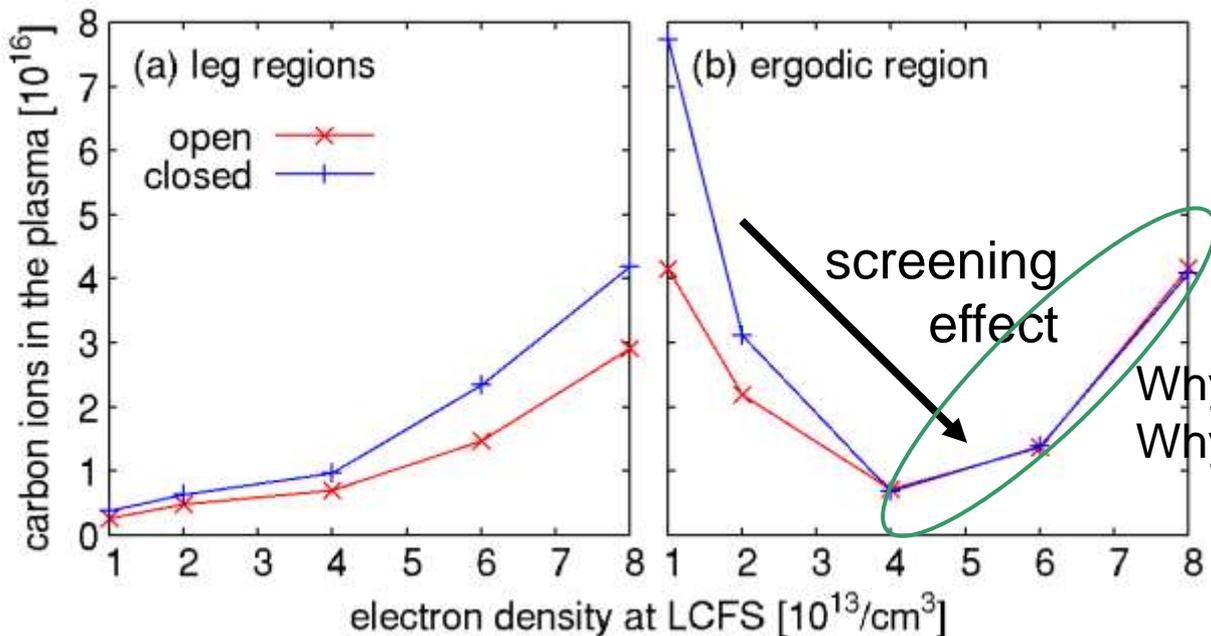
- ◆ Assumptions for qualitative comparison
 - Sputtering yield is fixed to 1%.
 - Self sputtering is not included.
 - Sputtered carbon is treated as an atom.
- ◆ Large recycling flux leads to large erosion.
- ◆ Modification to the closed configuration causes large erosion.
- ◆ Amount of the generation \neq Accumulation in the plasma





Impurity retention in the plasma

- ◆ Total carbon amount, i.e. $[C^+] + [C^{2+}] + \dots + [C^{6+}]$
 - Leg regions
 - Impurity retention increase according to the electron density.
 - Ergodic region
 - Screening effect of the plasma flow significantly reduces the retention in the ergodic region.
 - No influence of configurations is observed in the case of high-density discharges. — Why?



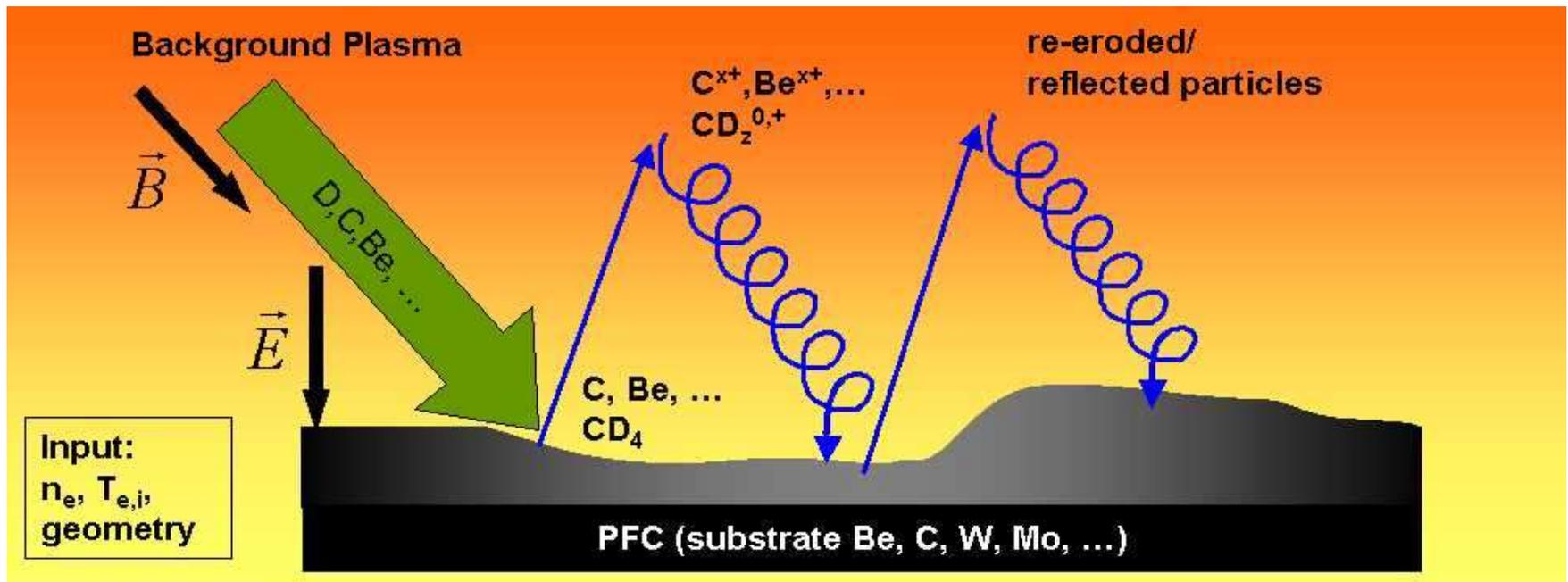


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ERO code

- ◆ Monte Carlo simulation code of erosion and redeposition
- ◆ Background plasma distribution is fixed in simulation.
- ◆ Code modification to use EMC3-EIRENE results is on going.





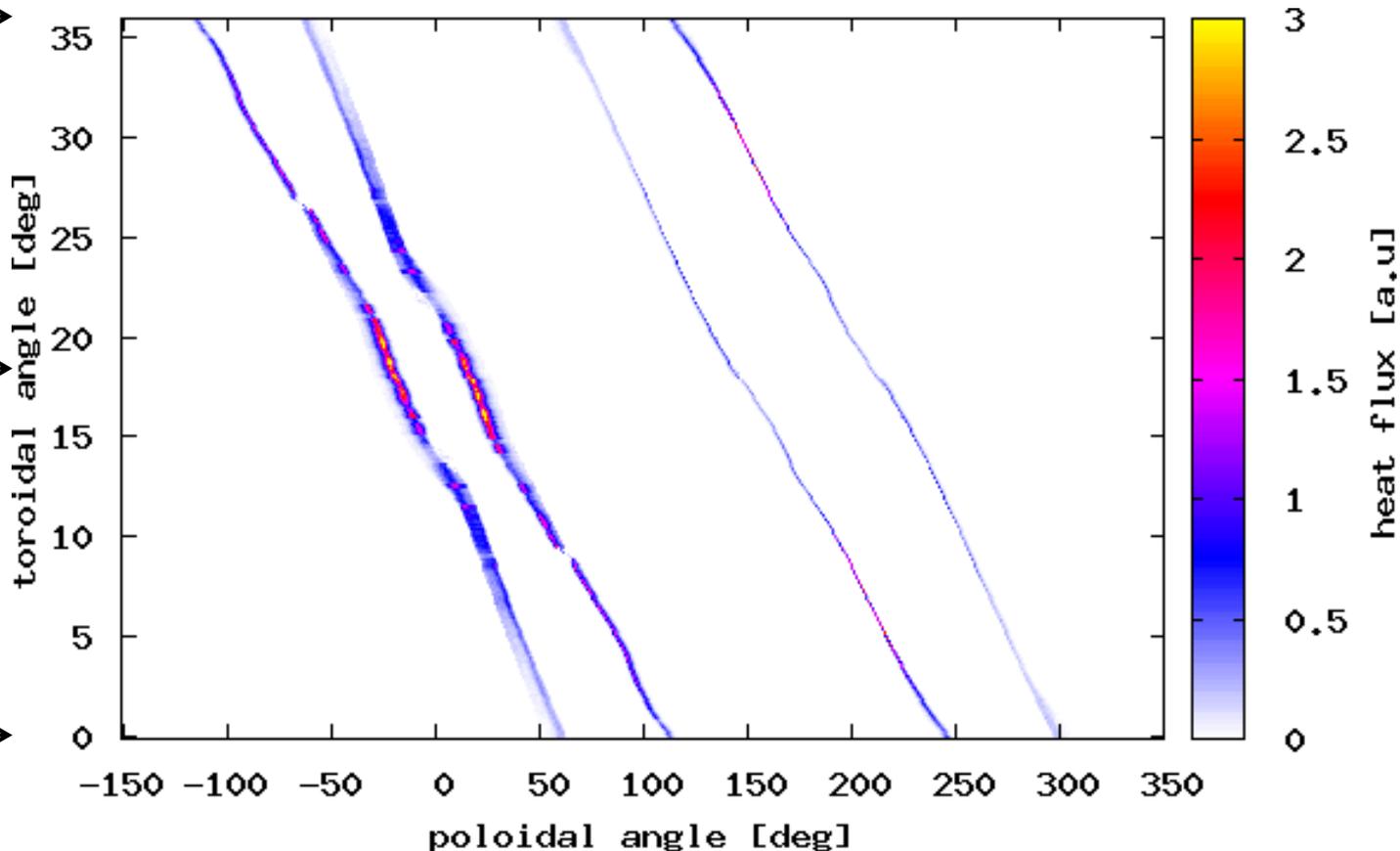
Heat load pattern

- ◆ EMC3-EIRENE results of heat load on divertor tiles.
 - Global estimation of erosion.
 - Influence of plasma parameters on the pattern.

vertically
elongated
plane →

horizontally
elongated
plane →

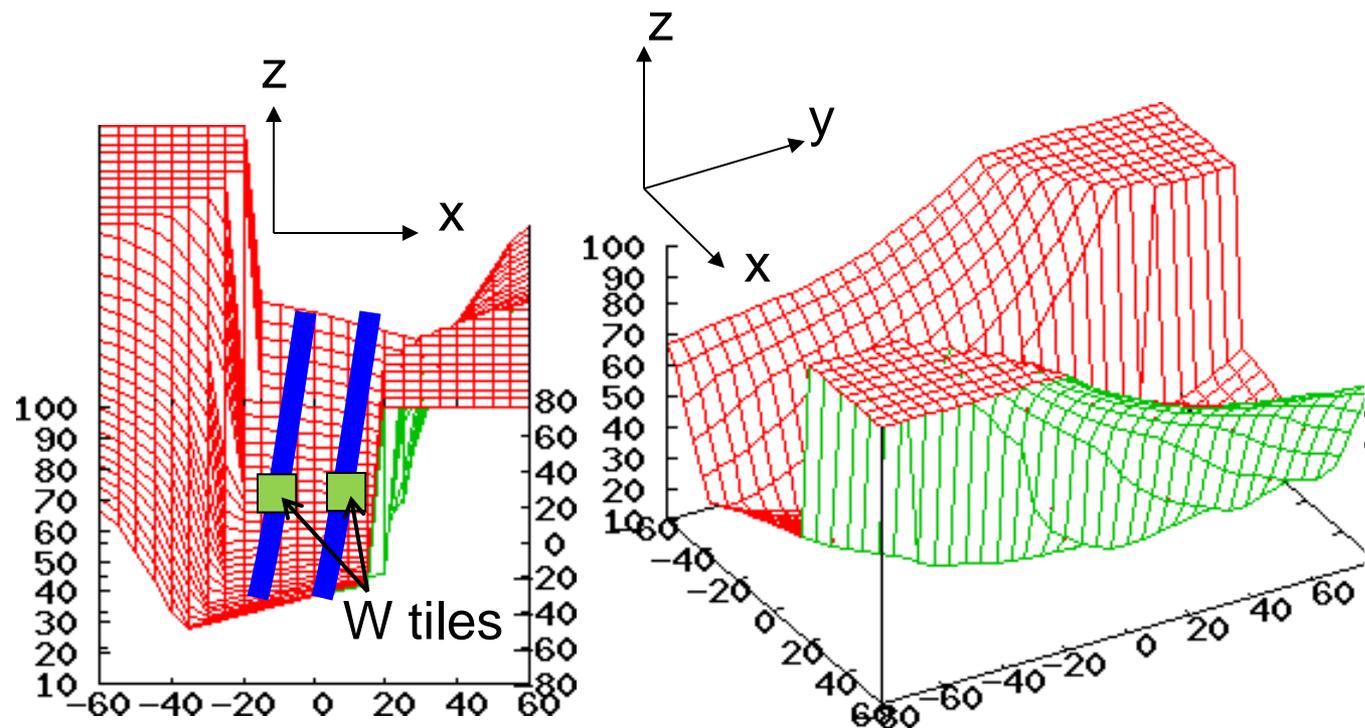
vertically
elongated
plane →



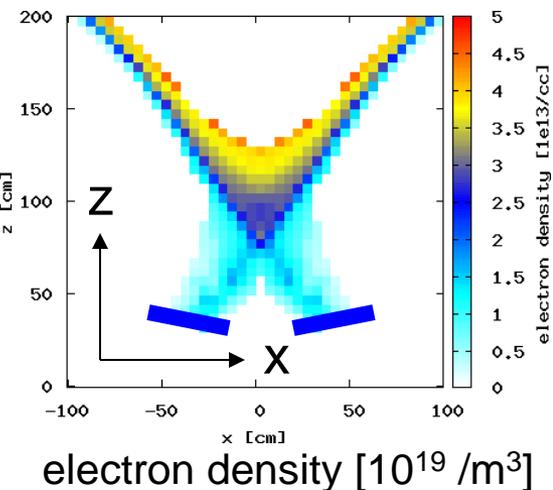


ERO simulation modeling

- ◆ Background plasma from EMC3-EIRENE
- ◆ How poloidal/toroidal plasma-distribution affects deposition/erosion of W.



simulation box of ERO





W measurement in LHD

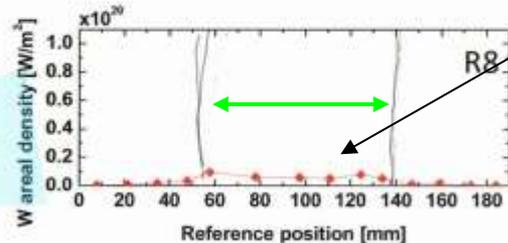
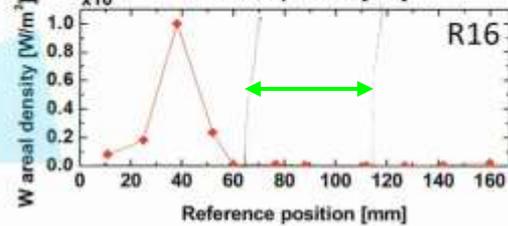
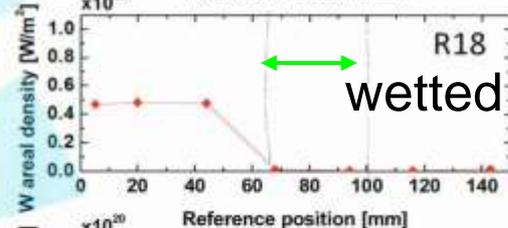
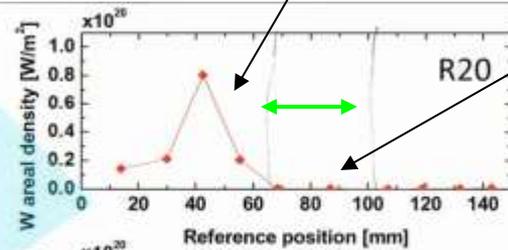
- ◆ A W-coated tile is installed.
- ◆ Other tiles are made of CFC.

W-coated tiles



W-deposition only in the private side

erosion dominant



wide deposition distribution



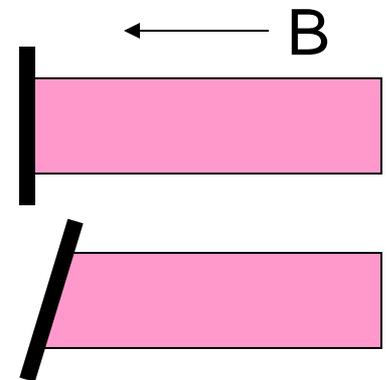
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Simulation modeling of linear device

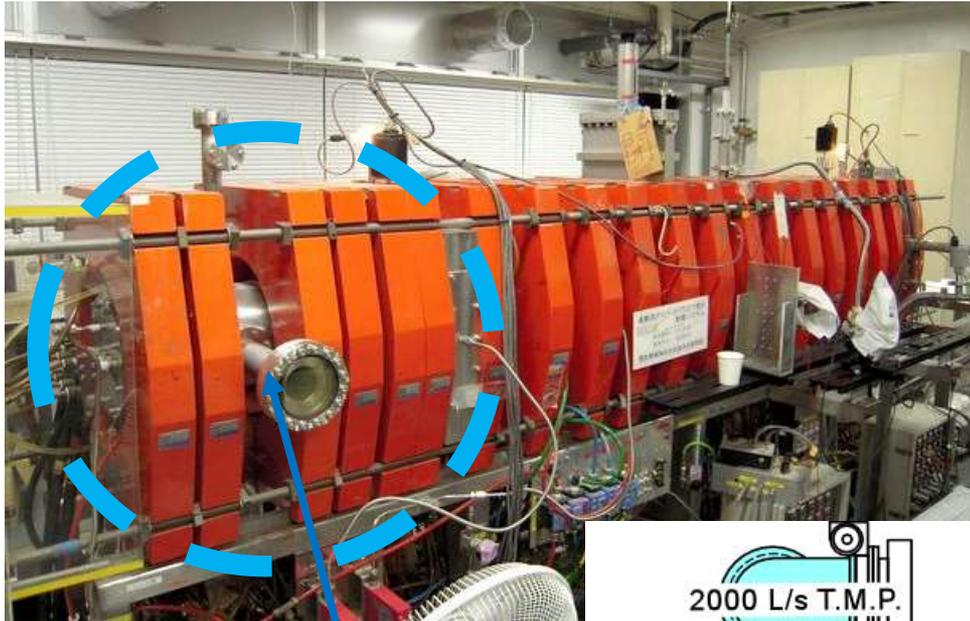
H. Tanaka, M. Kobayashi, G. Kawamura (NIFS)
N. Ohno, T. Kuwabara, M. Urakawa (Nagoya Univ.)
Y. Feng (Max-Planck IPP)

- ◆ Linear devices for divertor simulation and PWI
 - GAMMA10, NAGDIS-II, MAGNUM-PSI, PISCES-B, PILOT-PSI, PSI-2, JULE-PSI etc.
- ◆ Divertor-plasma codes have been developed for tokamak/helical devices.
 - 2D: B2-EIRENE (SOLPS), EDGE2D-NIMBUS, UEDGE-DEGAS2
 - 3D: EMC3-EIRENE
- ◆ Simulation of linear device has been reported but relatively new and challenging issue.
 - B2-EIRENE
 - Cylindrical plasma and device
- ◆ Advantage of EMC3-EIRNE code
 - 3D geometry of plasma and walls can be simulated.
 - Realistic geometry without cylindrical assumption.





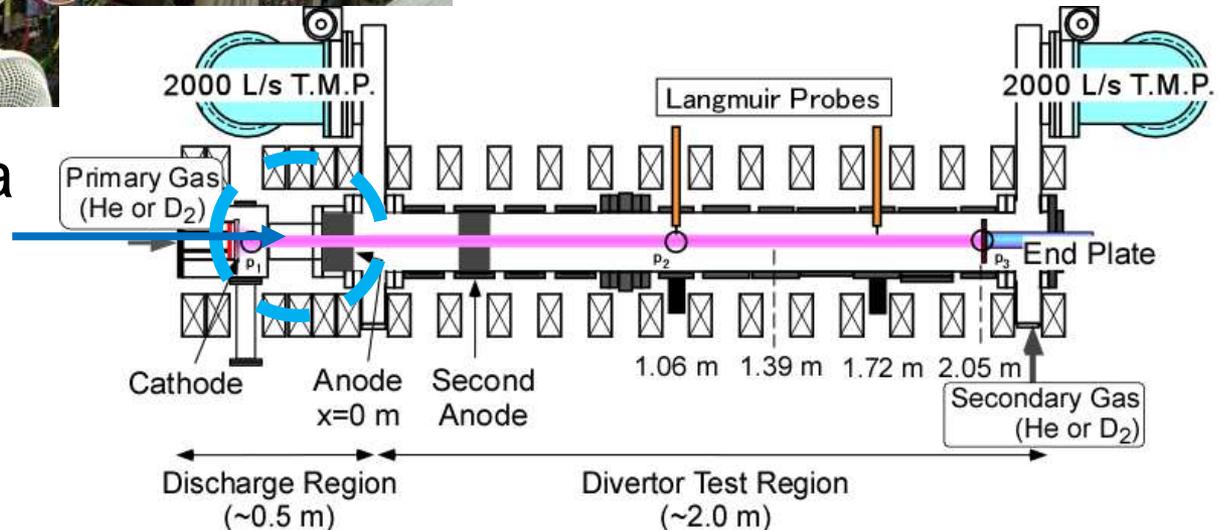
◆ Linear Device for PSI and detachment



uniform magnetic field
(< 0.25 T)
steady state DC discharge
(hours)

plasma parameter
 $n_e < 10^{20} \text{m}^{-3}$, $T_e < 10 \text{eV}$

plasma source

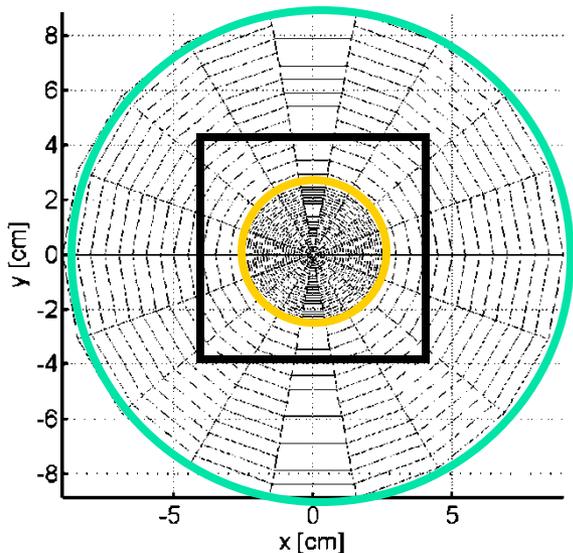
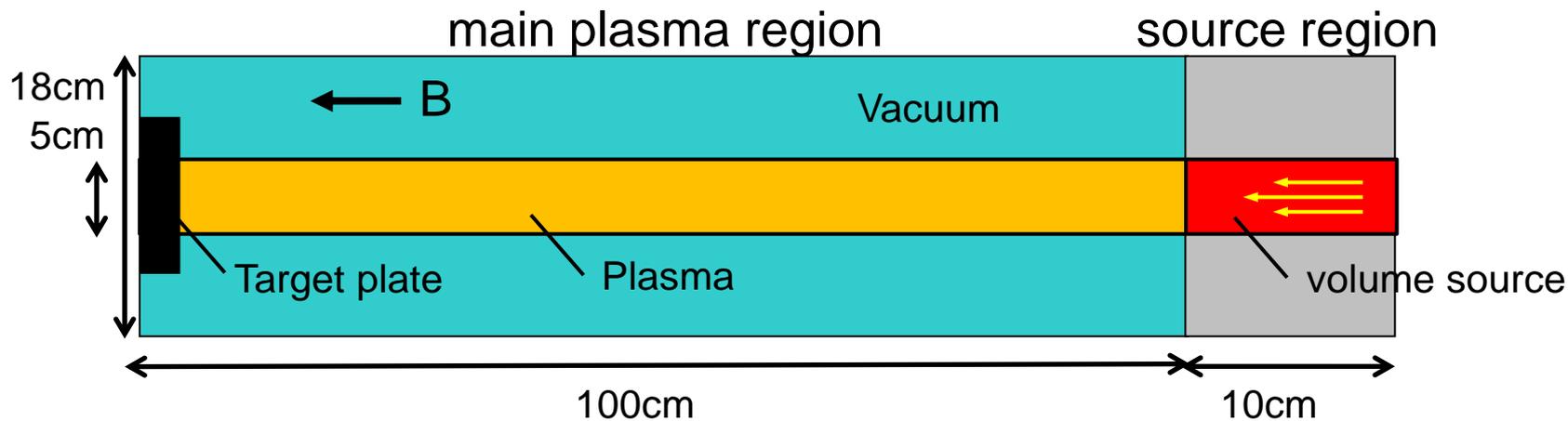


N. Ohno, *et al.*, Nucl. Fusion **41** (2001) 1055.



Modeling of linear plasma

◆ Development is in progress.



- ◆ Cylindrical mesh system.
- ◆ Wall and target plates with 3D shape
 - V-shaped target in future
- ◆ Detachment studies
 - NAGDIS-II
- ◆ PWI studies
 - NAGDIS-II, GAMMA10



Summary of the recent progress

- ◆ EMC3-EIRENE code and calculation mesh have been extended to LHD with the closed divertor configuration.
- ◆ A validation by neutral gas pressure
 - Influence of dome structure in closed configuration was analyzed.
 - The scaling of neutral pressure to $n_e(\text{LCFS})$ is in good agreement.
- ◆ Impurity screening effect in the open/closed configuration.
 - Screening effect due to the flow is recovered in the ergodic region.
 - Increase of impurity generation in the closed configuration.
- ◆ Application to ERO simulation
- ◆ Modeling of a linear device



Future plans

- ◆ Neutral particles
 - Pumping under the dome, wall pumping, gas-puffing
 - Penetration to the core (TASK3D)
- ◆ Impurities
 - Divertor configuration
 - Transport related to screening
- ◆ PWI
 - Local simulation at LHD W tile (ERO)
 - Global erosion estimation
- ◆ Measurements
 - Transport coefficient modeling from Thomson scattering
 - 2D imaging of impurity radiation
- ◆ Detachment
 - LHD (impurity seeding, RMP)
 - NAGDIS-II (1D)